



GENXFIN – Advanced nuclear energy for near-term future



SAFIR2018 Interim Seminar
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Timeline of Adoption

Safety

Economics

Licenciability in Finland

Sodium-cooled fast reactor (SFR)

- EBR-I (1951-1964), Sodium Reactor Experiment (1957-1964), Fermi 1 (1963-1966), EBR-II (1965-1994), Rapsodie (1967-1983), BN-350 (1973-1999), Phénix (1973-2009), PNR (1974-1994), Jōyō (1977-2007), BN-600 (1980-), Fast Flux Test Facility (1980-1992), FBTR (1985-1998), Superphénix (1980-), Monju (1994-1995, 2010-2010), BN-800 (2016-), China Experimental Fast Reactor (2011-)

Lead-cooled fast reactor (LFR)

- BM-40A, OK-550

High temperature gas-cooled reactor (HTGR)

- Dragon (1965-1976), Peach Bottom 1 (1966-1974), Fort St. Vrain (1979-1989), AVR (1967-1988), THTR-300 (1983-1989), HTTR (1999-), HTR-10 (2003-)

Molten-salt reactor (MSR)

- ARE (1954-1954), MSRE (1965-1969)

IAEA SMR Timeline



Immediate Deployable

Under Construction

CAREM-25

CNEA, Argentina

KLT-40S

OKBM Afrikantov, Russian Federation

HTR-PM

INET, People's Republic of China

Near-term deployable

Certified or at Advanced Design Stage

SMART

KAERI, Republic of Korea

RITM-200

OKBM, Russian Federation

PRISM

GE-Hitachi, USA

PBMR-400

PBMR, South Africa

BREST300-OD

NIKIET, Russian Federation

4S

Toshiba, Japan

ACP100

CNNC, People's Republic of China

NuScale

NuScale Power, USA

mPower

B&W, USA

GTHTR300

JAEA, Japan

SVBR-100

AKME Engineering, Russian Federation

ABV-6M

KMBM, Russian Federation

Mid to Longer-term Deployable

Conceptual Design for Future Deployment

AHWR300

BARC, India

Flexblue

DCNS, France

IRIS

IRIS International Consortium

DMS

Hitachi-GE, Japan

IMR

MHI, Japan

VVER-300

OKB Gidropress, Russian Federation

Westinghouse SMR

Westinghouse, USA

SMR160

Holtec, USA

VK-300

NikiET, Russian Federation

TH-100

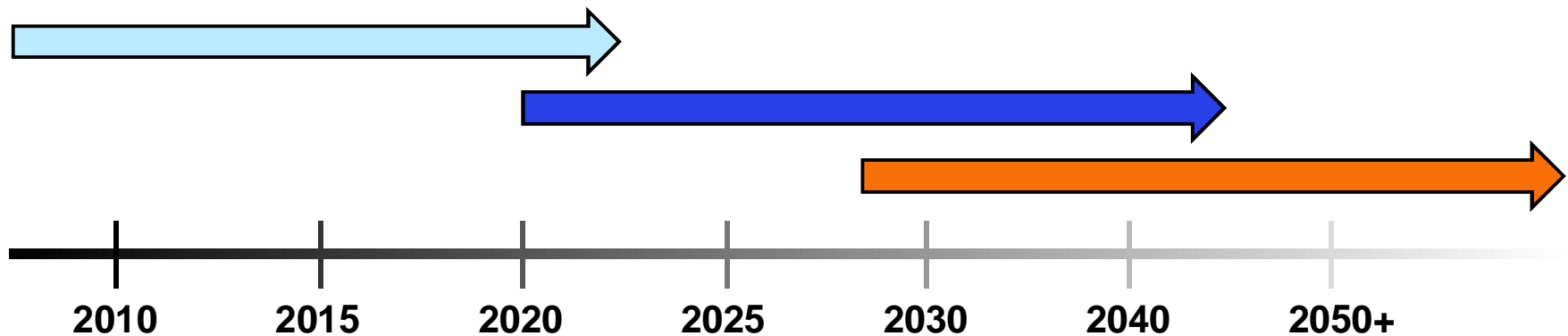
STL, South Africa

SC-HTGR

Areva, France

G4M

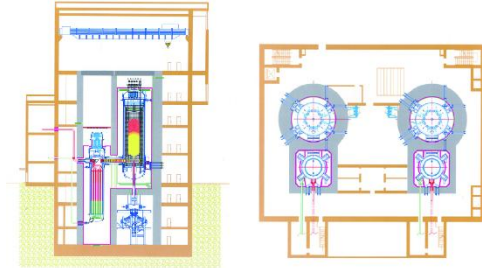
Gen4 Energy, USA



HTR-PM is a twin-reactor modular pebble bed HTGR demonstration plant. It is under construction at Huaneng Shidao Bay site, People's Republic of China and will connect to grid within a year.

Design highlights:

- 420.000 fuel elements in one reactor core
- 250 MW_{th} / 100 MWe per reactor unit (**40 % efficiency**)
- **567 °C** secondary circuit steam output at 13.25 Mpa
 - primary circuit helium at 750 °C at core outlet
- **Modular** on the scale of the power plant
 - Multiple reactor units for single turbine
- Passive safety: fully passive capacity for decay heat removal
 - Heat is conducted to structure: no need for natural circulation
- No central graphite column, no helium turbine, small core size
 - Avoids several problems otherwise associated with such designs



Prototype HTR-10 reactor was used to demonstrate the reactor behavior by **initiating several ATWS events**. The reactor shut itself down safely with no damage in all cases.

Similar tests with HTR-PM are expected this year to prove the design's inherent safety.

HTR-PM is a first-of-a-kind demonstration plant. Its **actualized costs** per MWe exceed corresponding traditional LWR costs by **10-20%** before cost reductions from mass production or establishment of HTGR industry.



TECHNOLOGY «FOR» BUSINESS

